

# Sorption of $^{237}\text{Np}$ by $\text{UO}_2$ under Repository Conditions

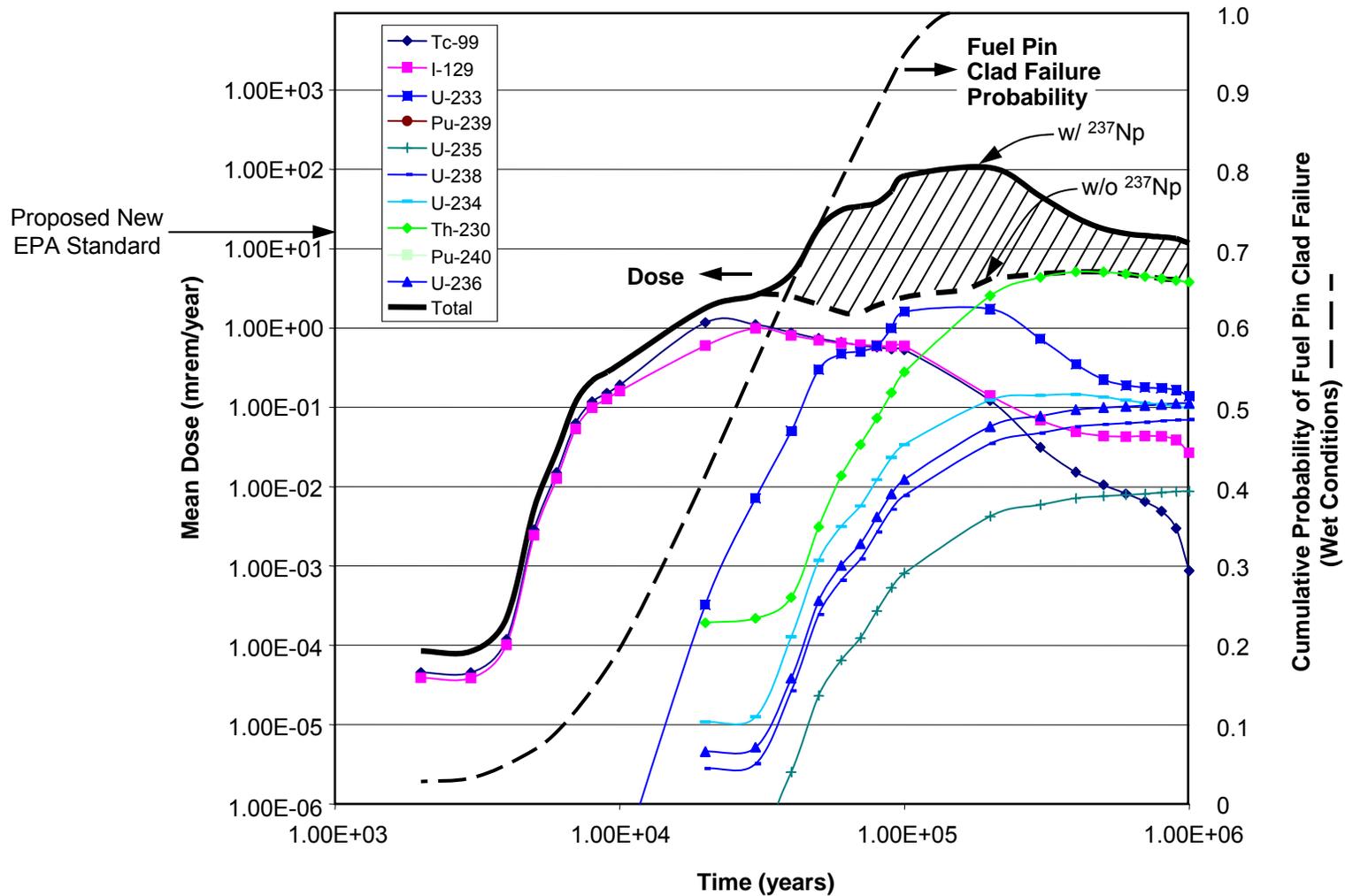
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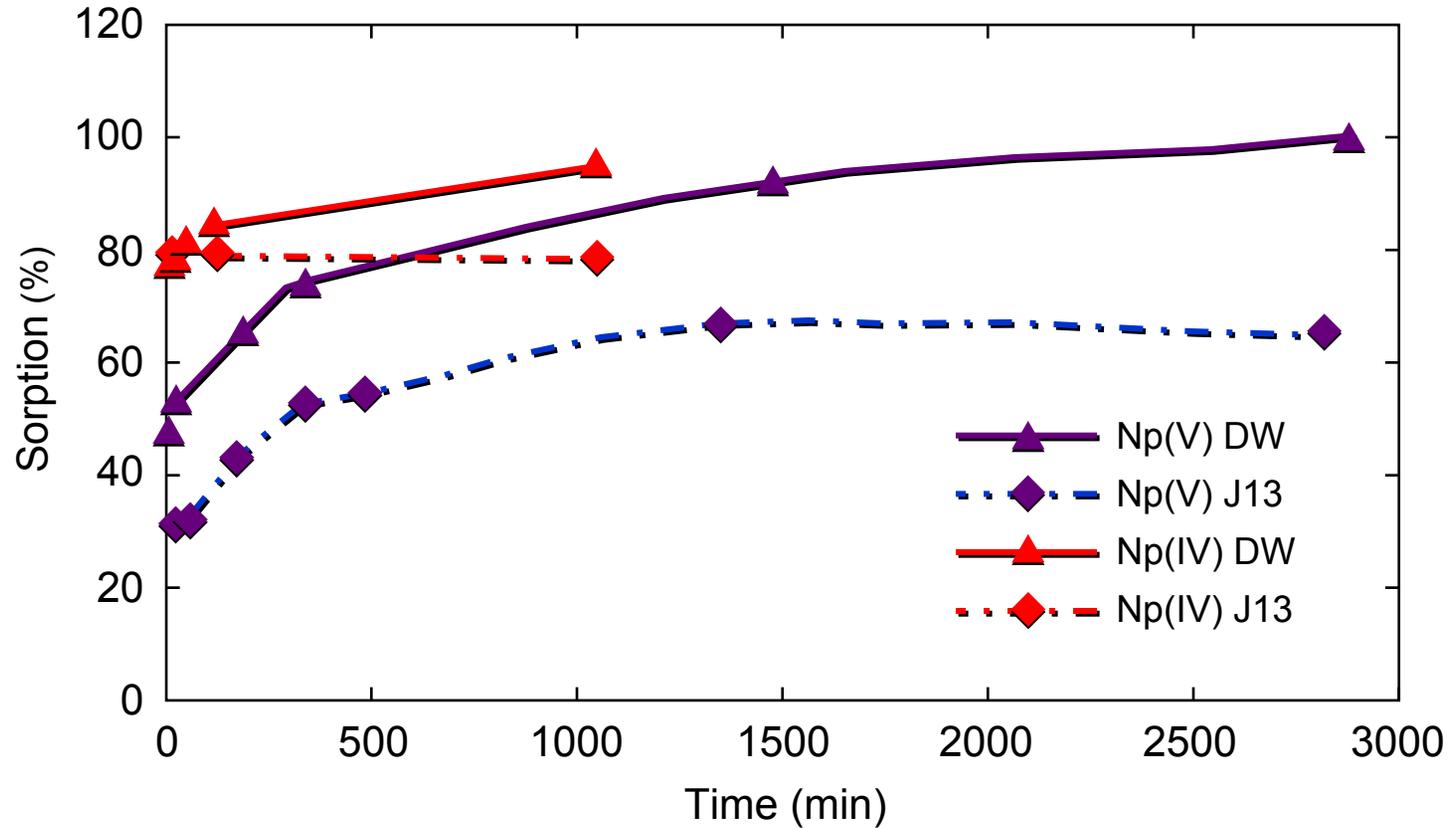
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Conference  
Las Vegas, Nevada  
April 30–May 4, 2006

# The Contribution of $^{237}\text{Np}$ to Total Site Boundary Calculations

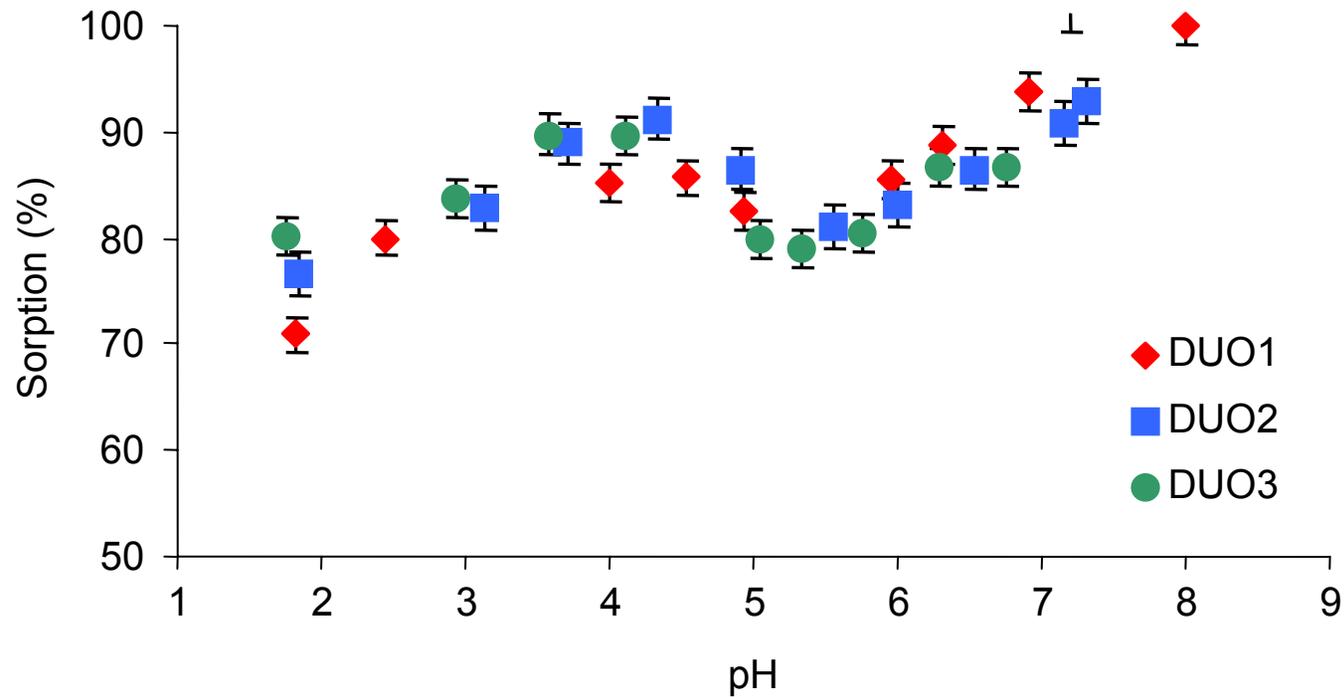


(Ref. Scientific and Technical Priorities at Yucca Mountain, EPRI Report No. 1003335, December 2003).

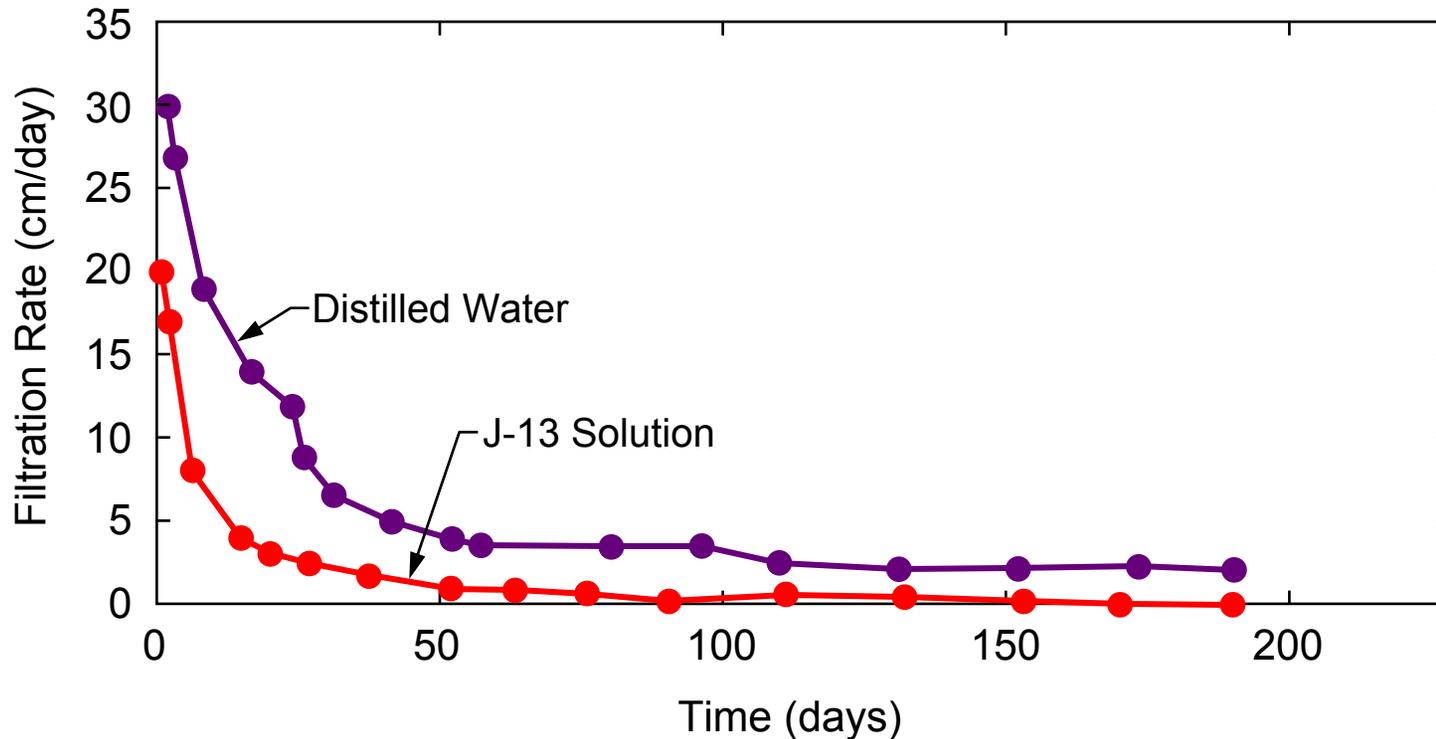
# Neptunium Sorption from Aqueous Solutions onto Uranium Dioxide



# Np(IV) Sorption onto $\text{DUO}_2$ vs pH



# Relatively Thin Layers of $\text{UO}_2$ Become Impermeable to Water



\*Data courtesy of T.V. Kazakovskaya (VNIIEF, Russia) and E.V. Zaharova (IPC, Russia)

# Some Key, General Conclusions of Russian/American Sorption onto $\text{UO}_2$ Research

- **$\text{UO}_2$  sorbs Np(V) and Np(IV). Equilibrium is reached rapidly.  $\text{UO}_2$  pretreatment temperature affects the amount sorbed. Increase of temperature to  $95^\circ\text{C}$  increases the amount sorbed. Np(V) sorption is irreversible**
- **TC(IV) sorbs onto  $\text{UO}_2$**
- **Filtration of water flow through  $\text{UO}_2$  is almost complete in a short period of time**

# Experiment Data Implications

- **UO<sub>2</sub> may be an effective chemical barrier to key radionuclide release**
- **Spent nuclear fuel burn-up is ~<5%. The remaining 95% UO<sub>2</sub> may retain key radioisotopes**
- **Radionuclide retention might be enhanced by using depleted uranium (DU) oxides in a Richard's Barrier (i.e., as backfill), using DUO<sub>2</sub>-steel cermets as a material of construction for waste packages, filling the void space within the waste canister with DUO<sub>2</sub>, using DUO<sub>2</sub> as ballast in the invert**

# Potential DU Dioxide Applications in a Geologic Repository

